



Agenzia nazionale per le nuove tecnologie,
l'energia e lo sviluppo economico sostenibile



Attività ENEA sulla fusione nell'ambito della collaborazione con INFN

Presentato da M. Ciotti NUC-PLAS
In collaborazione con Fabio Panza



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Outline

1. Lithium Technology for IFMIF-DONES
2. Contributo del Laboratorio NIXT (ENEA) ai programmi INFN-E e INFN n-ToF
3. Contributi ENEA all'ICRH per DTT
4. Laser-initiated p+11B fusion reactions
5. Reattori ibridi fusione-fissione per la produzione di trizio
6. ENEA-INFN sulla neutronica

Lithium Technology for IFMIF-DONES

M. Tarantino, D. Bernardi, P. Favuzza, G. Micciché, F.S. Nitti
Dipartimento Nucleare

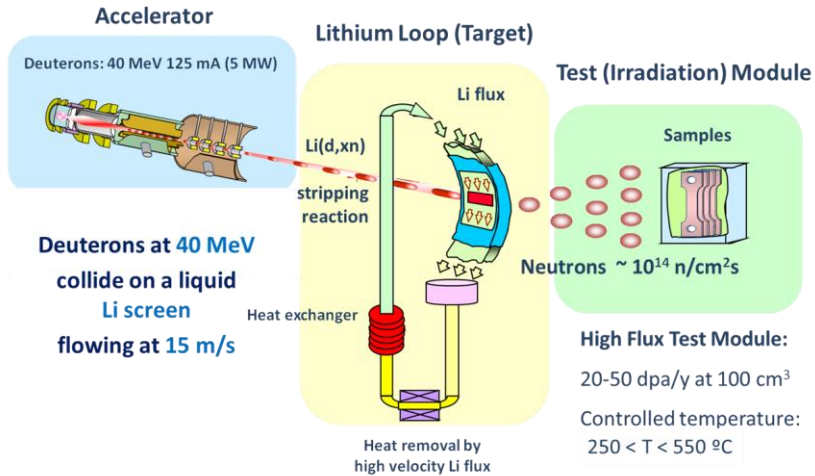


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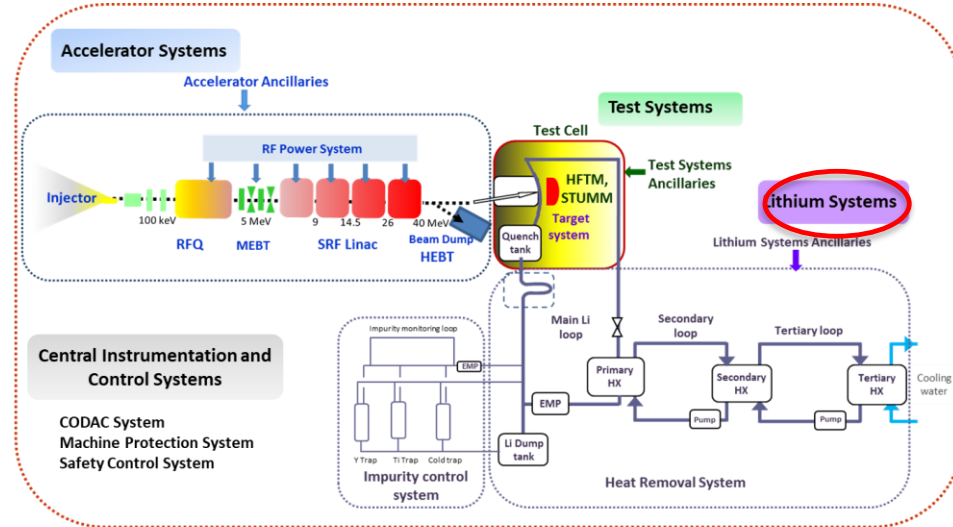


IFMIF-DONES plant configuration

An accelerator based fusion-like neutron source required for the qualification of the materials to be used in the EU DEMO



A neutron flux of $\sim 10^{14}$ cm⁻²s⁻¹ is generated with a neutron spectrum up to 50 MeV energy



Site, Buildings & Plant Systems

Layout & Site Infrastructures

Buildings

HVAC, Electrical Power Supply, HRS, etc.

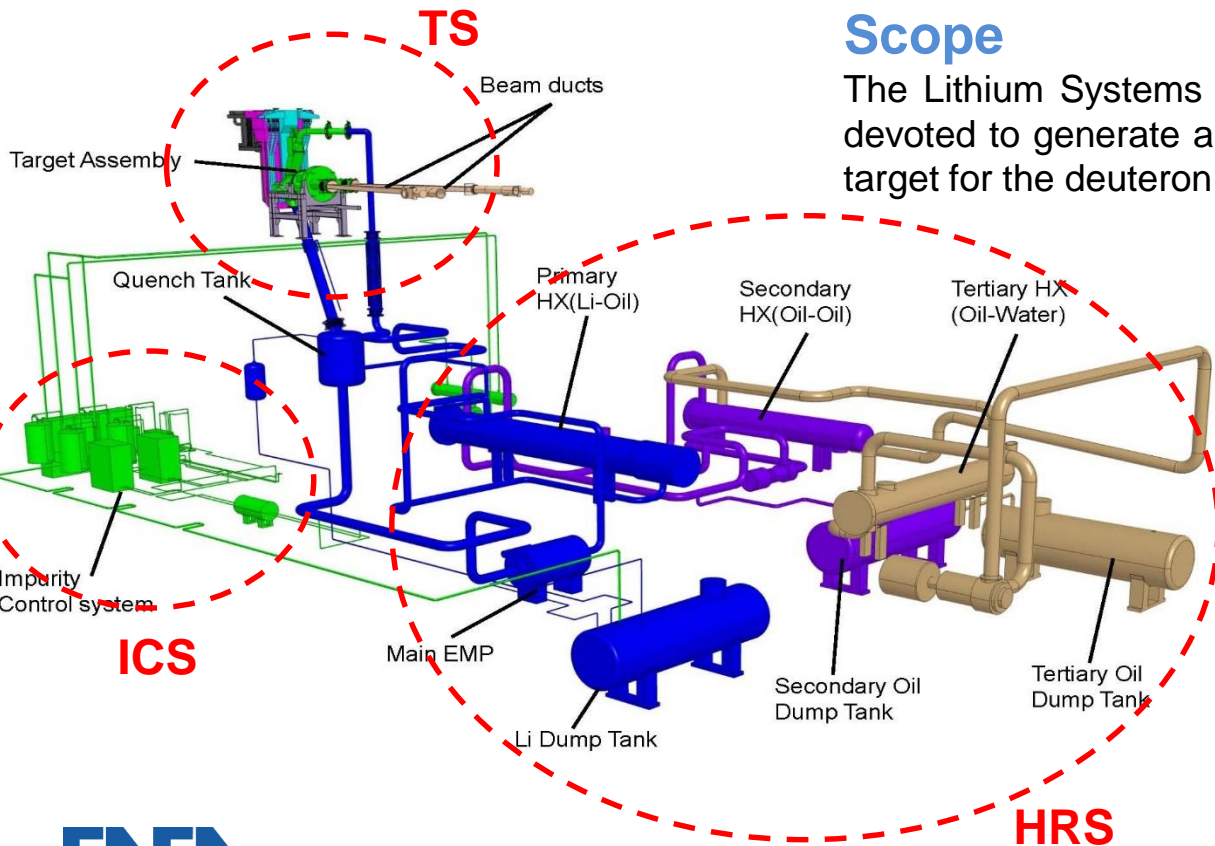
Remote Handling System

Central Control Systems and Integrated Instrumentation

Lithium Systems overview

Scope

The Lithium Systems (LS) in the IFMIF-DONES plant are devoted to generate and maintain a stable flowing liquid Li target for the deuteron beam interaction



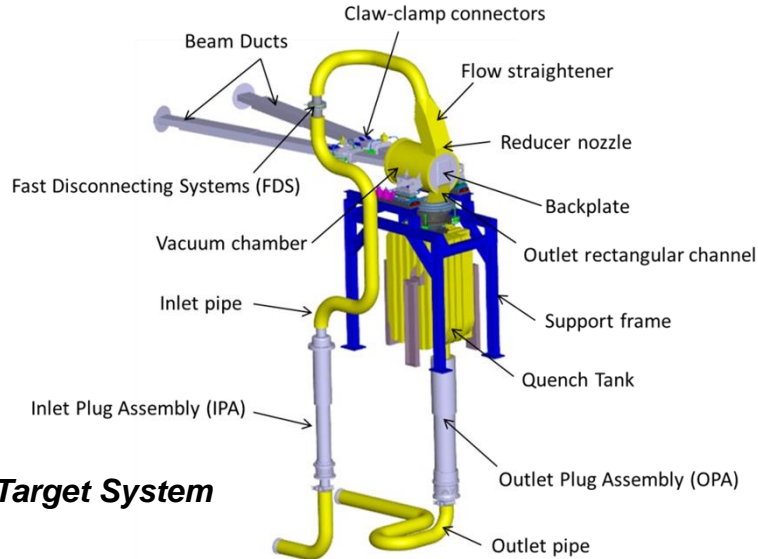
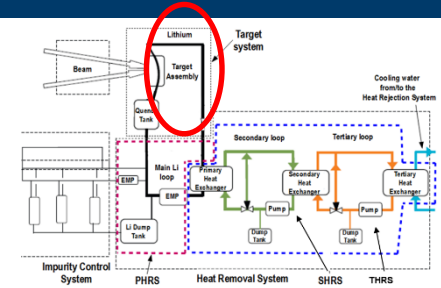
Four subsystems can be identified:

- *Target System (TS)*
- *Heat Removal System (HRS)*
- *Impurity Control System (ICS)*
- *Ancillary Systems (LSA)*

Target System

The Target System is the core of the plant, it has two main functions:

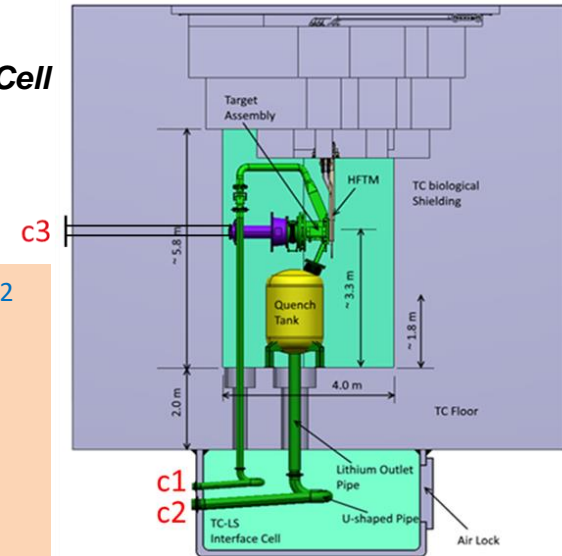
- Stop the deuteron beam, by means a flowing lithium target, and generate the neutron field for the High-Flux Test Module (HFTM) irradiation
- Remove the thermal power deposited by the beam



Target System



Test Cell



Heat flux: 500 - 1000 MW/m²

Li jet thickness: 25 ± 1 mm

Li inlet T: 300 °C

Li velocity: 15 m/s

Chamber pressure: 10⁻² Pa

Main material: EUROFER-97

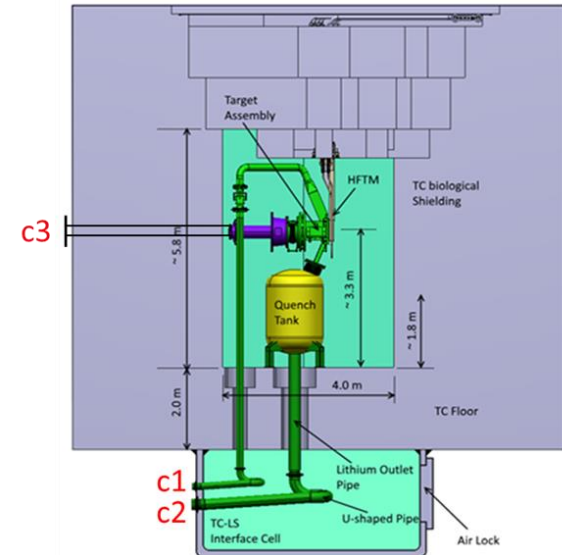
Target System

Very challenging system:

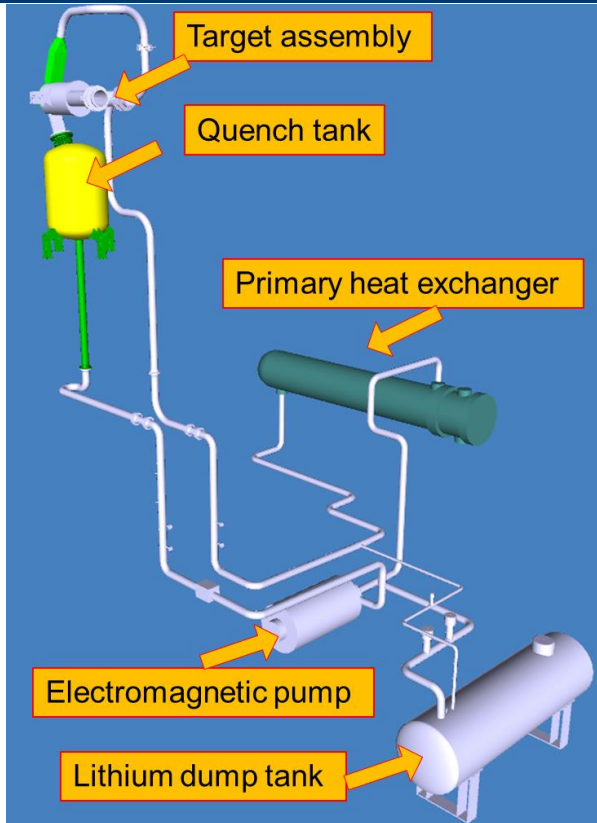
- High heat flux (up to 1 GW/m^2)
- Strong nuclear (neutron+gamma) radiation
- RH compatible
- High vacuum operation
- Extremely accurate tolerances and dimensional precision
- High availability (1 fpy of continuous irradiation)

Design well advanced!

Main contribution from ENEA during the past IFMIF-EVEDA project and EUROfusion WPENS FP8



HRS - Primary Li Loop



The **Primary Loop** is the main circuit of the Heat Removal System whose function is to remove the thermal power produced by the beam in the Target Assembly. Lithium, flowing in the Primary Loop, transfers its heat duty to the Secondary Loop through the “Primary Heat Exchanger” (Li-oil HX)

ITEMS	SPECIFICATIONS
Fluid	Liquid Lithium
Design temperature	350°C
Design pressure	0.76 MPa (g)
Maximum Flow Rate	0.104 m ³ /s
Piping Flow Speed of Lithium	< 6 m/s
Material	Stainless Steel 316L
Pipe nominal diameter	6"
Pipe internal diameter	154.1 mm
Pipe schedule	40

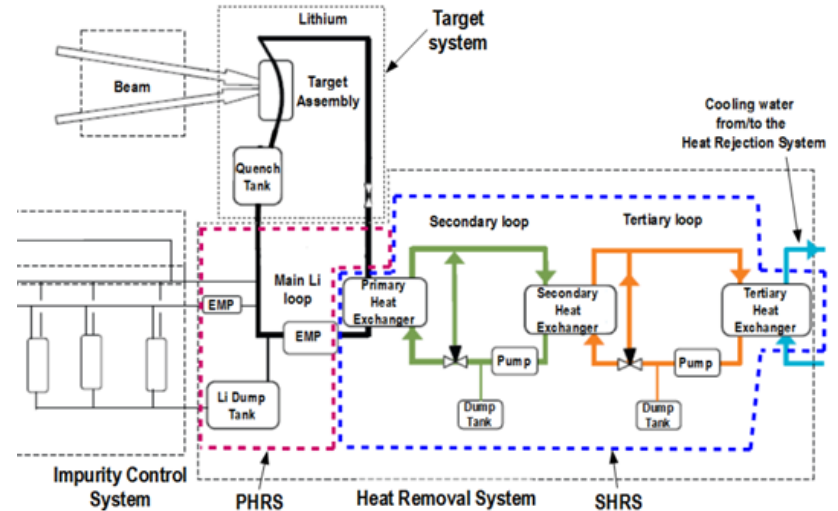
Update design is being developed by a EU Industrial Consortium (Ansaldo Nucleare, Empresarios Agrupados, Framatome).

HRS - Secondary and Tertiary Loops

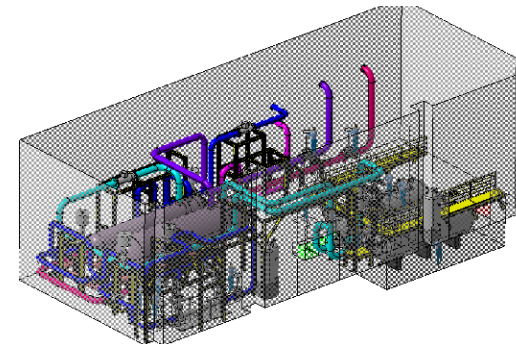
These subsystems are devoted to evacuate the thermal power deposited in the lithium by the D+ beam towards the general water cooling system of the plant

Two oil loops in series are foreseen:

- Secondary oil loop incl. Li-oil & oil-oil HXs
- Tertiary oil loop incl. oil-water HX



No direct contribution from ENEA/Italy in the past but it might be of future potential interest for the Italian industry



Overall view of the layout

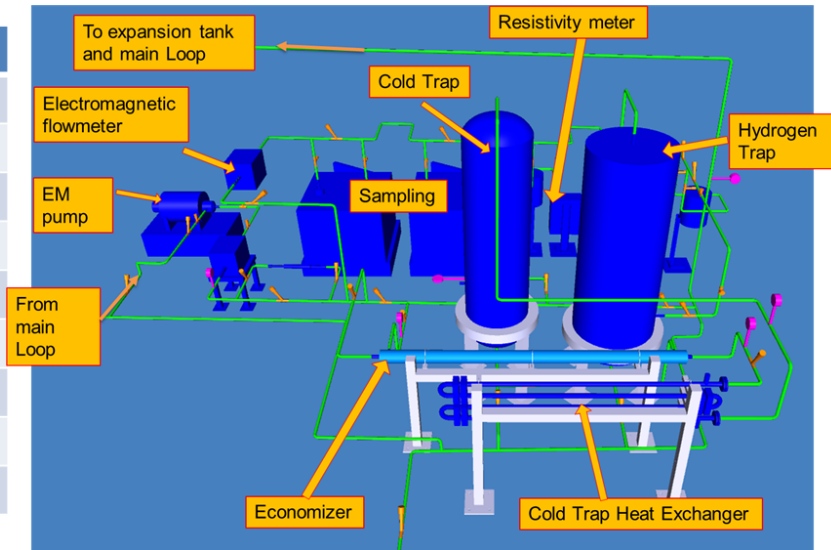
Impurity Control System (ICS)

The **Impurity Control System** is a branch circuit of the Primary Lithium Loop and it is designed to fulfil the following functions:

- Confine the sources of radioactivity such as **Tritium, Beryllium-7 and activated corroded/eroded metallic elements**
- Monitor and control **impurity levels in the Li loop** (including O, C, N, Be, H, T etc...)

The ICS consists of a main loop - **Impurity Control Loop** - and one daughter sub-loop, the **Impurity Monitoring Loop**; the first is dedicated to impurity removal, by means of a Hydrogen Trap and a Cold Trap, while the second is a sampling/analysis system.

ITEMS	SPECIFICATIONS
Fluid	Liquid Lithium
Design temperature	350°C
Design pressure	0.16 MPa (g)
Maximum Flow Rate	0.65 l/s
Material	Stainless Steel 316L
Pipe nominal diameter	3/4"
Pipe internal diameter	20.9 mm
Pipe schedule	40
Pipe total length	≈ 80 m



ENEA is involved since a long time in the definition of the lithium purification procedures; Li sampling and off-line analyses; corrosion/erosion studies (Lifus 6 loop)

Update design of ICS is being developed by a EU Industrial Consortium (ANN, EAI, FA).

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Contributo del Laboratorio NIXT (ENEA) ai programmi INFN-E e INFN n-ToF

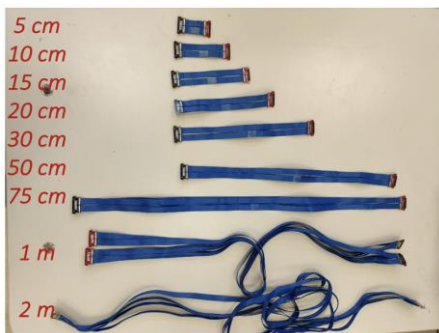
G. Claps, D. Pacella, F. Cordella, V. De Leo, A. Tamburrino, V. Piergotti

Contributo del Laboratorio NIXT (ENEA) ai programmi INFN-E e INFN n-ToF

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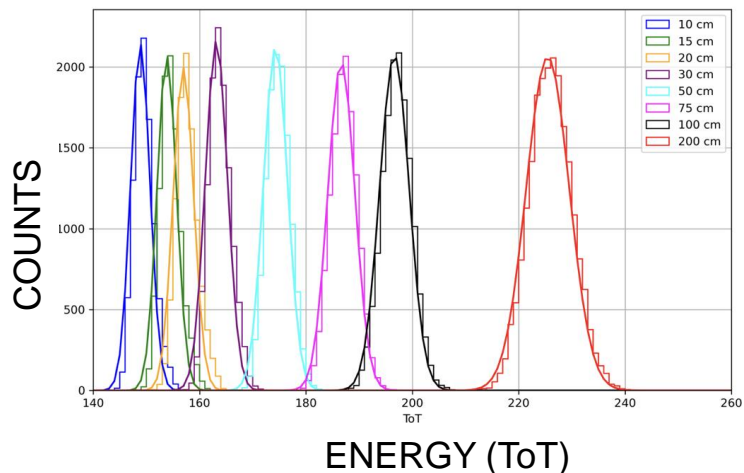
GEM Gas Detector for X-ray radial camera of ITER

Development of a GEM gas detector with remote front-end electronics for X-ray radial camera of ITER under high fusion neutron fluxes (10^7 n/s/cm²)



Cable lengths

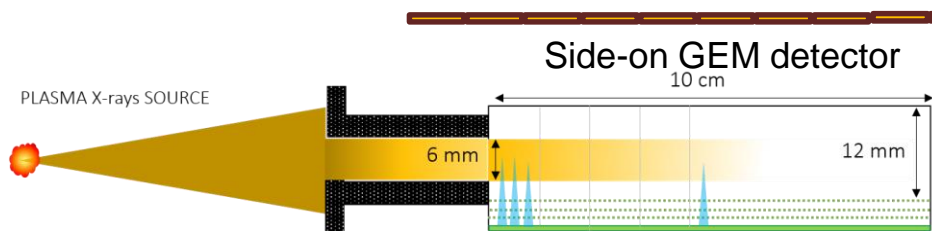
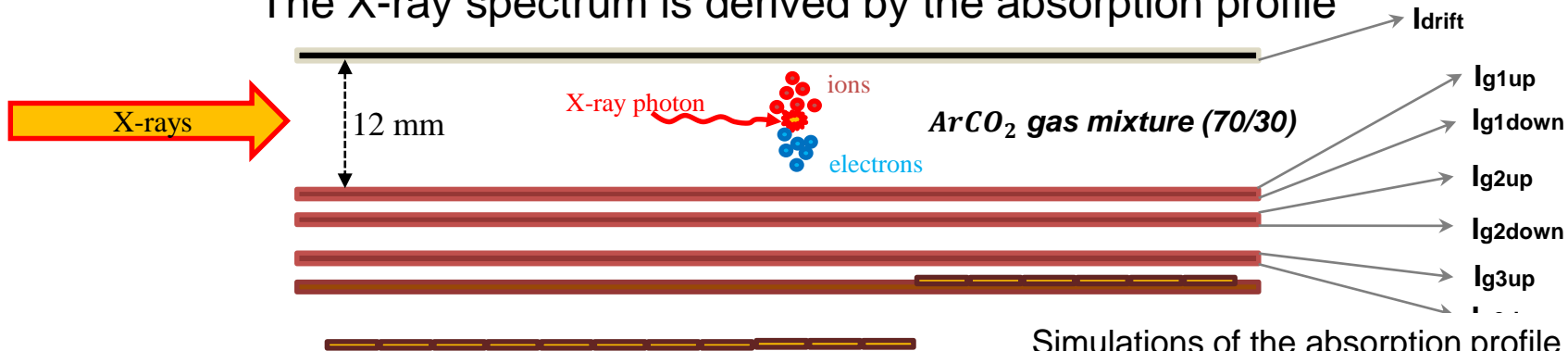
Response to a monoenergetic input



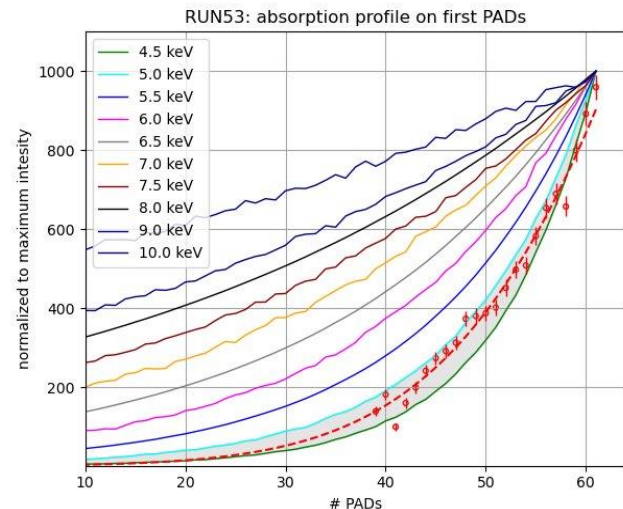
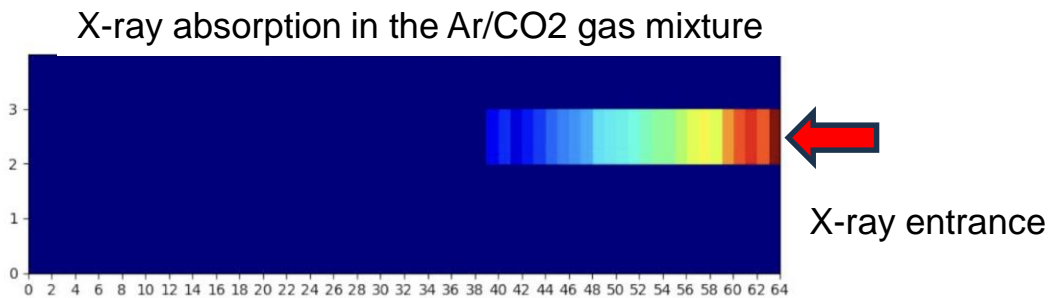
The side entrance X-ray GEM camera, with its f.e. electronics shielded and located 1-2 meters away, can operate at ITER in the 10-80 keV energy range, even with a 14 MeV neutron flux of 10^7 n/s/cm²

Soft X-ray spectroscopy for Laser Plasmas with side-on GEM detector

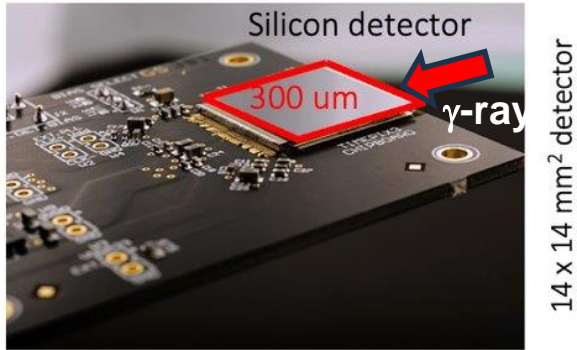
The X-ray spectrum is derived by the absorption profile



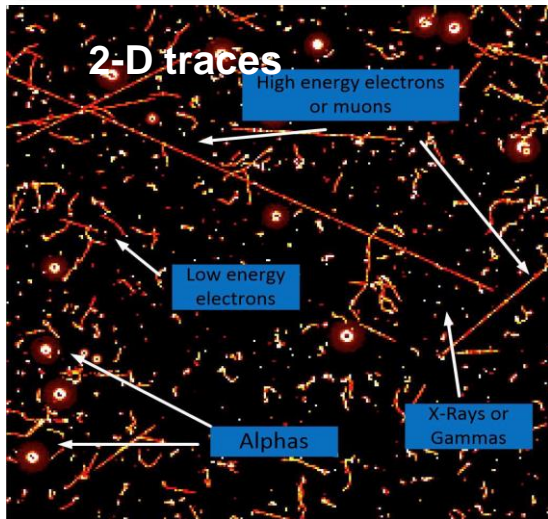
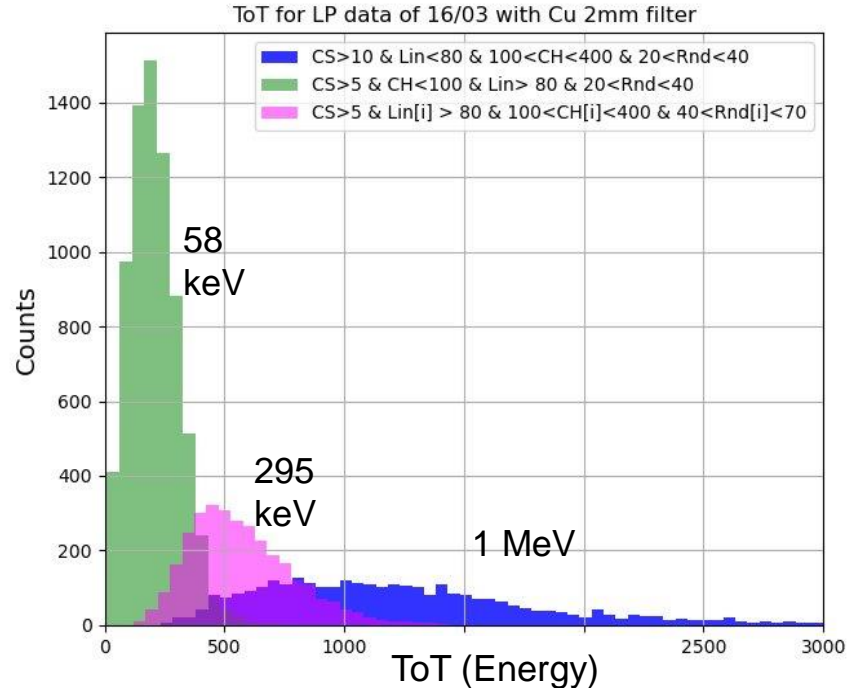
Simulations of the absorption profile with different temperatures of the ablated plasma



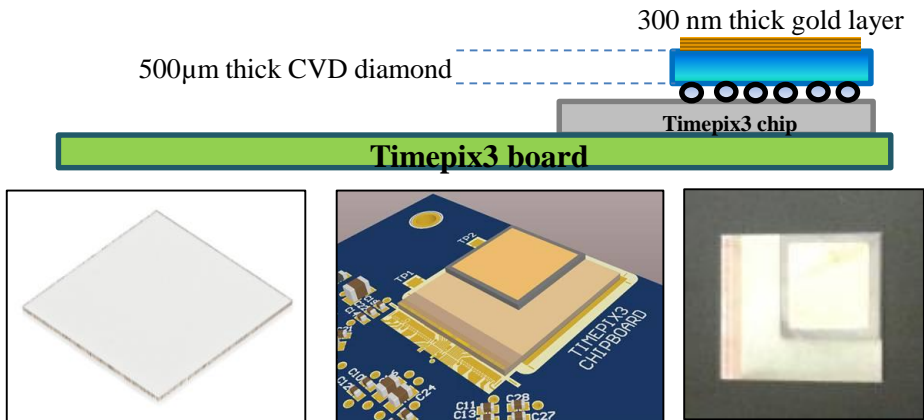
Gamma-ray diagnostic for Laser Plasmas with a side-on silicon Timepix3 detector by means of the 2-D morphology of the traces



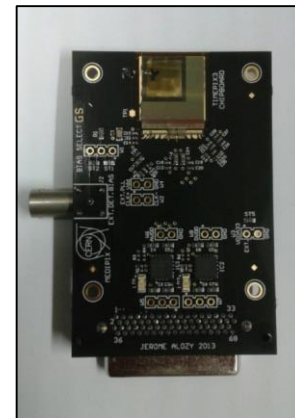
Two major gamma-ray emissions are observed at 58 keV and 295 keV, with a tail extending to 1MeV



Diamondpix detector for fast neutron detection in nuclear fusion



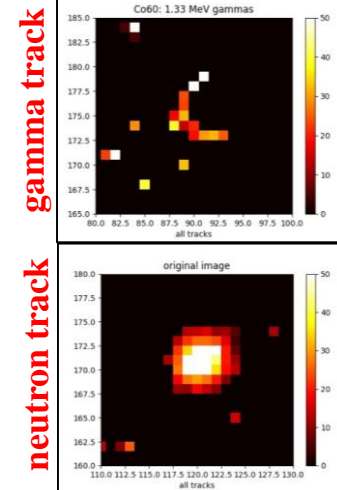
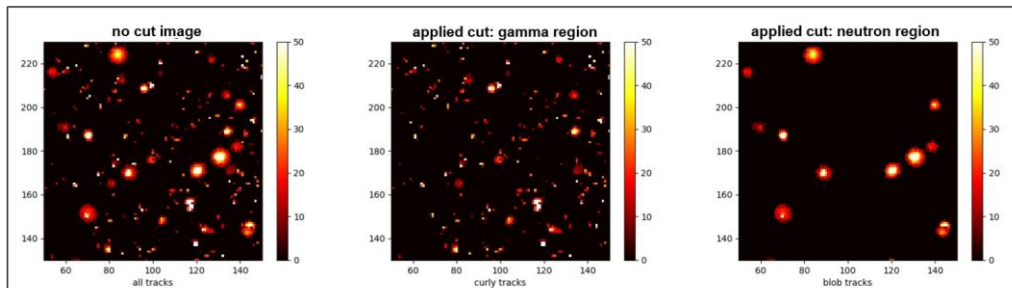
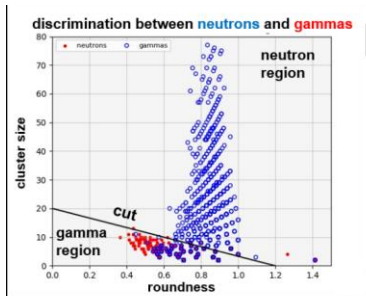
A CVD polycrystalline diamond has been bump-bonded to a Timepix3ASIC



FNG facility, ENEA Frascati

Neutrons discrimination against gammas

14 MeV neutrons

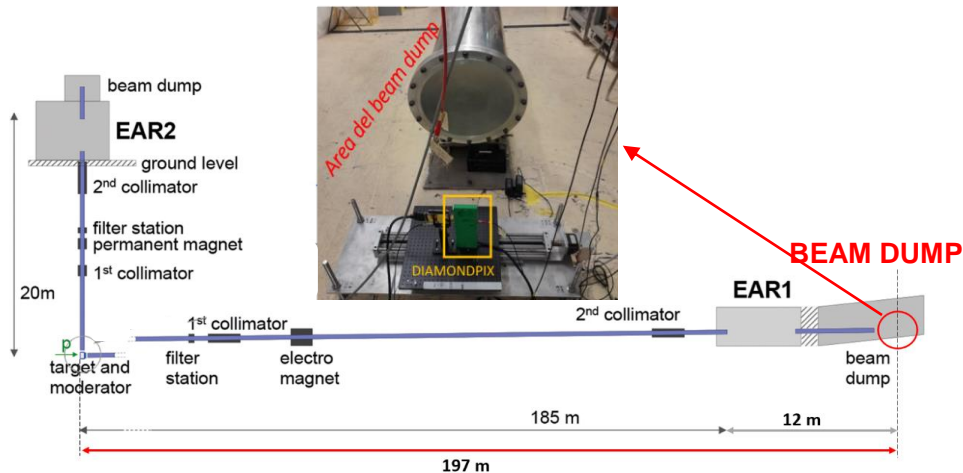


An algorithm has been developed to select tracks based on their morphological parameters and released charge



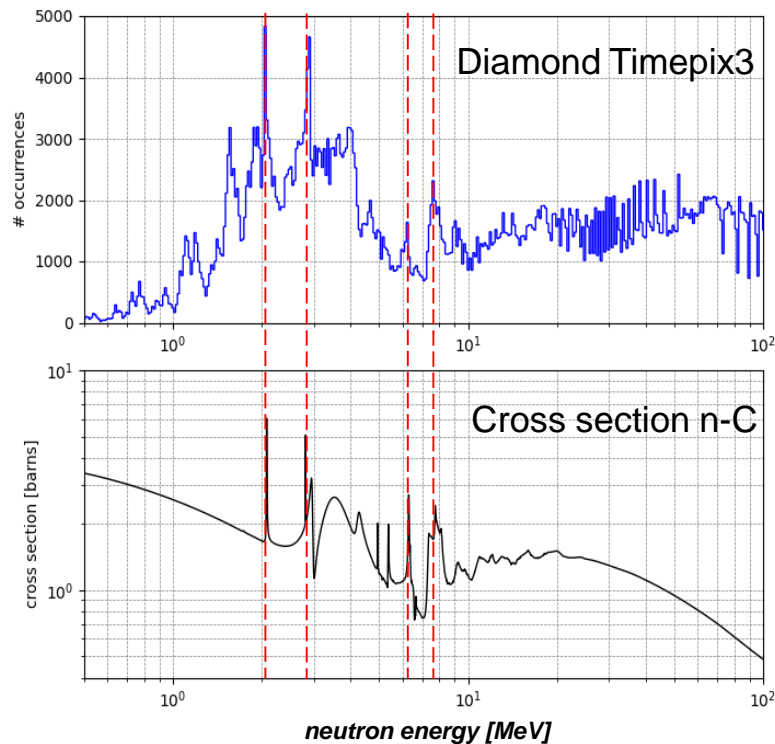
Time of flight measurements on the n_TOF facility (CERN)

Experimental set-up on the n_TOF facility



ACQUISITION PARAMETERS

- TPX3 was controlled by the **Katherine module**
- **DATA-DRIVEN MODE**
- **ACQUISITION MODE: ToT & ToA (charge and time)**
- **Acquisition time window: 150 ms (1 GeV – 10 meV)**



Several ToF peaks corresponding to carbon resonances are observed, validating the spectrum reconstruction and enabling the selection of the desired energy range

Contatti

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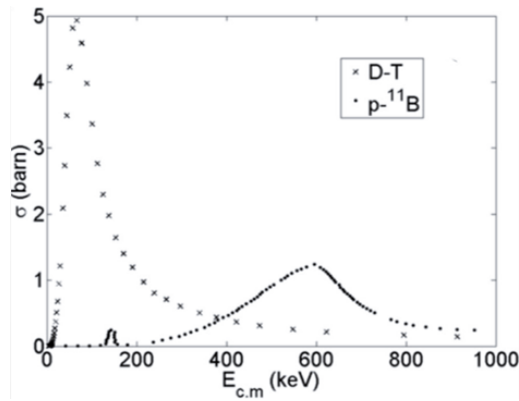
Laser-initiated $p+^{11}\text{B}$ fusion reactions

F. Consoli (ENEA), G. A. P. Cirrone (INFN)

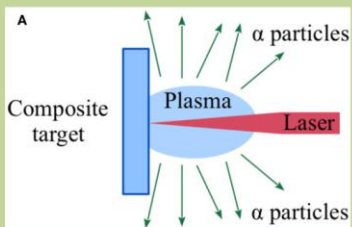
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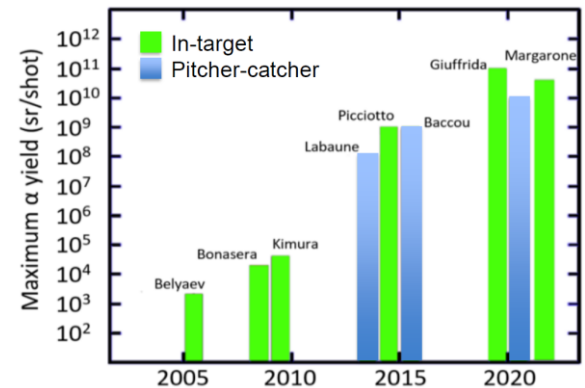
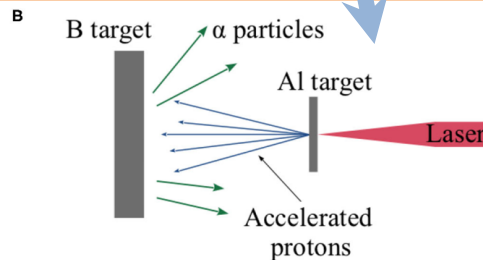
- $^{11}\text{B}(p,\alpha)2\alpha$ fusions by means of laser-plasma interaction demonstrated [1-2].
- A significant yield achieved \rightarrow potential future employment in non-thermal conditions for
 - energy production and other applications. Institutions and company involvement
- Study of low-rate nuclear reactions in plasmas also important for astrophysical researches
- Two main approaches. Scheme A: “in-target” and B “pitcher-catcher”, with energetic nanosecond and picosecond pulsed lasers.



Scheme A. H and B plasma by laser pulses on composite targets: i.e. B-doped plastic or Si enriched with H and B. ns and/or ps laser pulses.



Scheme B. Laser accelerated protons sent to a boron target or to a boron plasma. ps or fs laser pulses

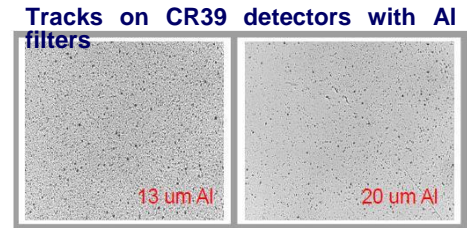
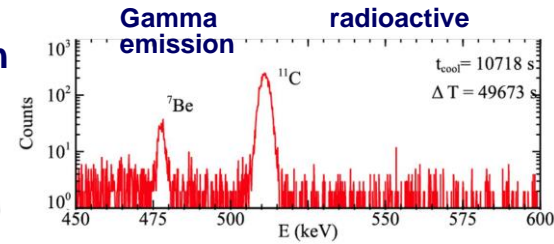
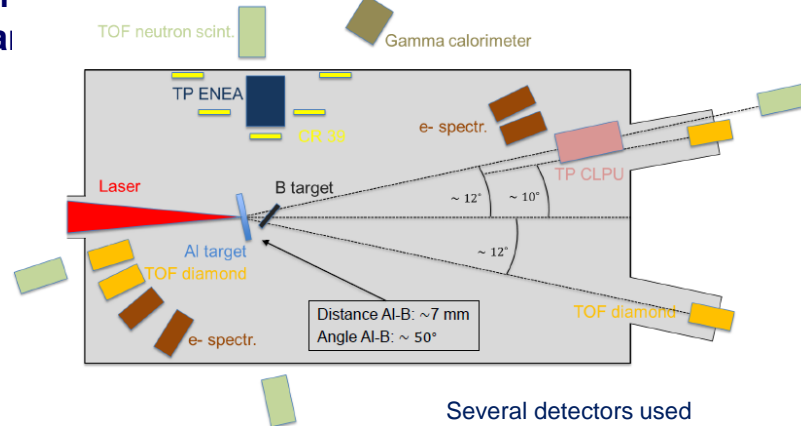
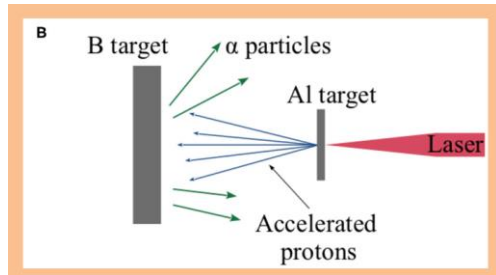


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INFN
 Istituto Nazionale di Fisica Nucleare

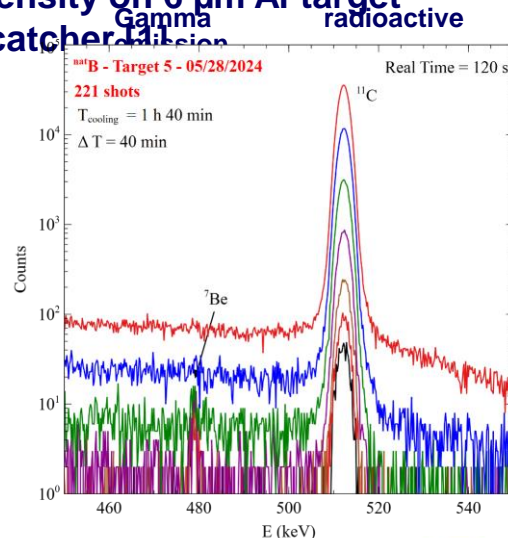
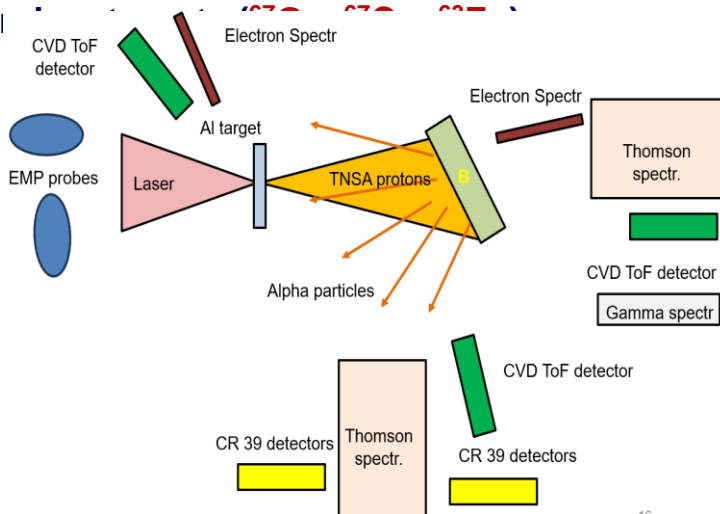
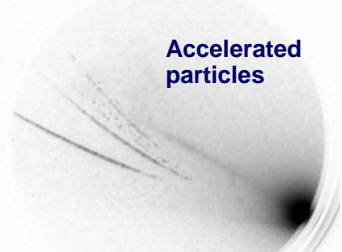
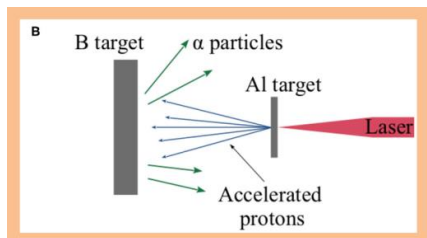
Pitcher-catcher experiments in high repetition rate femtosecond laser (1) - 2022

- Previous experiments with significant alpha yields were performed with **energetic and single pulse lasers**. **Alternative**: exploiting of high energy and **high repetition rate lasers**.
- Broad collaboration: ENEA (PI), INFN-LNS, Texas A&M University (US), University of Tor Vergata, University of Perugia, University of Pisa, University of Bordeaux (FR), Institute of Plasma Physics and Laser Microfusion (PL), Queen's University of Belfast (UK), ELI Beamlines (CZ), Institute of Plasma Physics of Czech Academy of Sciences (CZ)
- VEGA III Petawatt Laser, CLPU, Salamanca, Spain: 30 J, 200 fs, $5 \cdot 10^{19}$ W/cm² intensity on 6 μ m Al target
- Significant activation (¹¹C, ⁷Be) produced on ¹¹B, sign of p+¹¹B fusions in activation on secondary targets (⁶⁷Co, ⁶⁷Cu, ⁶³Zn)
- Clear signature of alpha pair



Pitcher-catcher experiments in high repetition rate femtosecond laser (2) - 2024

- Previous experiments with significant alpha yields were performed with **energetic and single pulse lasers**. **Alternative:** exploiting of high energy and **high repetition rate lasers**.
- Broad collaboration: ENEA (PI), INFN-LNS, Texas A&M University (US), University of Tor Vergata, Università di Milano Bicocca, University of Bordeaux (FR), Institute of Plasma Physics and Laser Microfusion (PL), ELI Beamlines (CZ), Institute of Plasma Physics of Czech Academy of Sciences (CZ)
- L3-ELIMAIA, ELI-BEAMLINES, Czech Republic: 10 J, 25 fs, $5 \cdot 10^{20}$ W/cm² intensity on 6 μ m Al target
- Significant activation (¹¹C, ⁷Be) produced on ¹¹B, sign of p+¹¹B fusions in B catcher
- Significant activation on second



The Italian “FUSION” initiative



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FUSion Studies of proton boron Neutronless reaction
in laser-generated plasma



ALMA MATER STUDIORUM
UNIVERSITÀ DI BOLOGNA



Financed by INFN,
Spokepersons: GAP Cirrone
(INFN) and F Consoli (ENEA)

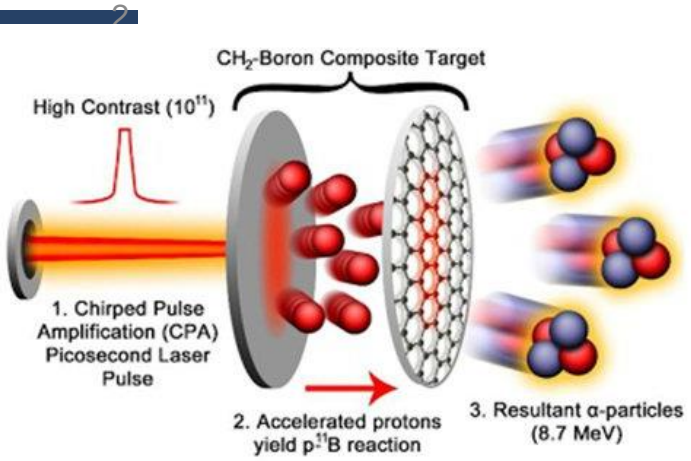


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- Laser induced $p\text{-}^{11}\text{B}$ reactions in both in plasma and pitcher-catcher configuration



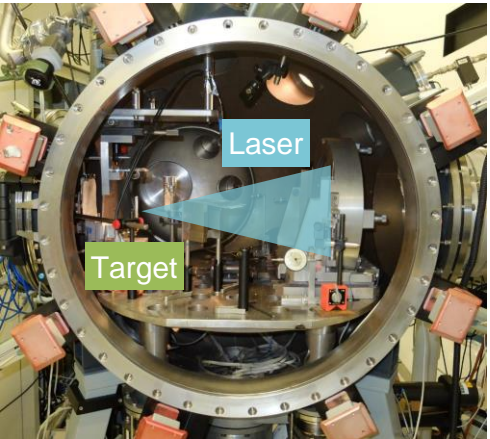
Basic understanding of fusion mechanism in plasma

Realisation of **new targets** for the $p11\text{B}$ reaction improvement

Development of **new diagnostics**

Maximisation of the **alpha yields** in plasma

Ion **stopping power** measurements in plasma and development of new computational approach



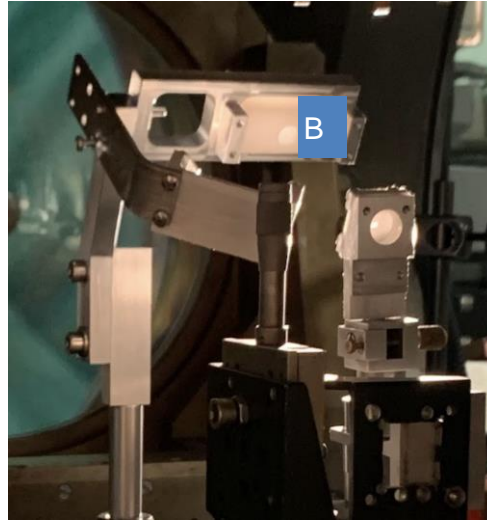
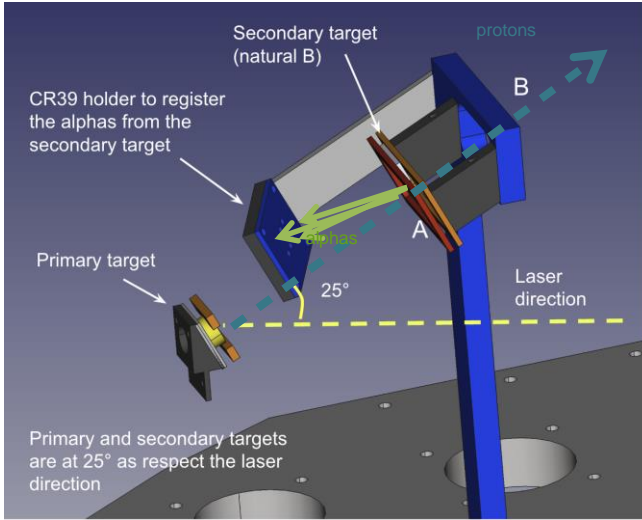
Consoli F et al., (2020) Front. Phys. 8:561492

Cirrone, Giuffrida et al. (2020) Phys Rev E 2020 Jan;101(1-1):013204.



The PALS interaction chamber (Prague, CZ)

FUSION first experiments, Feb 2024, PALS, Prague



Four different targets

Diagnostics: Time of Flight, Thomson Spectrometer, gamma detectors, electron spectrometers

6 MeV protons measured in backward

Boron activation measured

Analysis is ongoing

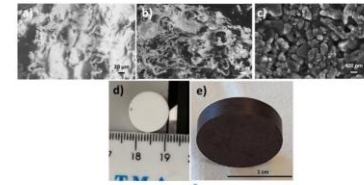
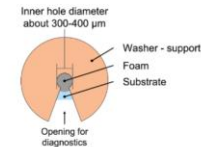


Figure 16. SEM pictures of target surfaces: a) reference; b) resin/boric acid 2:1 wt.; c) resin/boron 2:1 wt.; d) picture of a typical reference target; e) target of resin/boron 5:1



Contatti

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Contributi ENEA all'ICRH per DTT

S. Ceccuzzi (ENEA), D. Mascali (INFN)



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Design activities for the ICRH system of DTT



Solid-state transmitters

Transmission line & matching, feedthrough, integration

Antennas

task ITP



task ILP



INFN-LNS and ENEA contribute together to the design of DTT ICH antenna

task IAP



ENEA contribution to the ICRH system of DTT (1/4)

Enea contribution to this joint activity mostly concern:

- contribution to RF design and modelling,
- analysis & design of cooling circuit,
- antenna shape and diagnostics.

Contributors

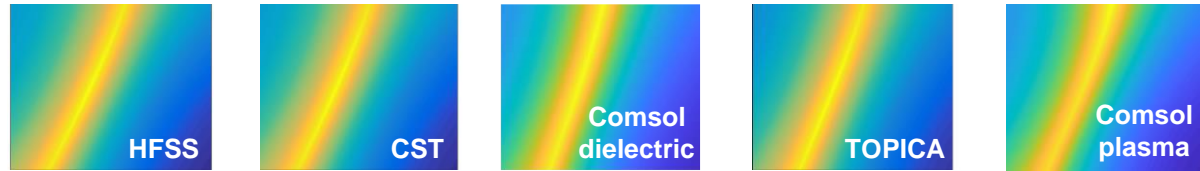
- Reference person at INFN: **D. Mascali**
- Reference person at ENEA: **S. Ceccuzzi**
- ENEA contributors: N. Badodi, A. Del Nevo, M. Eboli, F. Giorgetti, M. Iafrati, P. Maccari, R. Marinari, G.L. Ravera.

ENEA contribution to the ICRH system of DTT (2/4)

Enea contribution to this joint activity mostly concern:

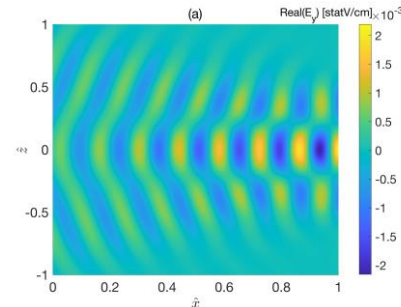
- contribution to RF design and modelling, →
- analysis & design of cooling circuit,
- antenna shape and diagnostics.

- Fruitful cooperation in the **design**, with development and successful benchmark of advanced design tools:



Coupled power [a.u.] vs. $P_{\text{cen}}/P_{\text{out}}$ and $\Delta\phi$ respectively varying in the ranges [0.1-10], [-40,40], with P_{cen} (P_{out}) = coupled power by central (lateral) straps, and $\Delta\phi$ = phasing deviation from 0π

- Strong interaction on the **modelling** of plasma waves, also thanks to visits of ENEA staff to INFN- LNS in 2021 and vice versa in 2023 and 2024.

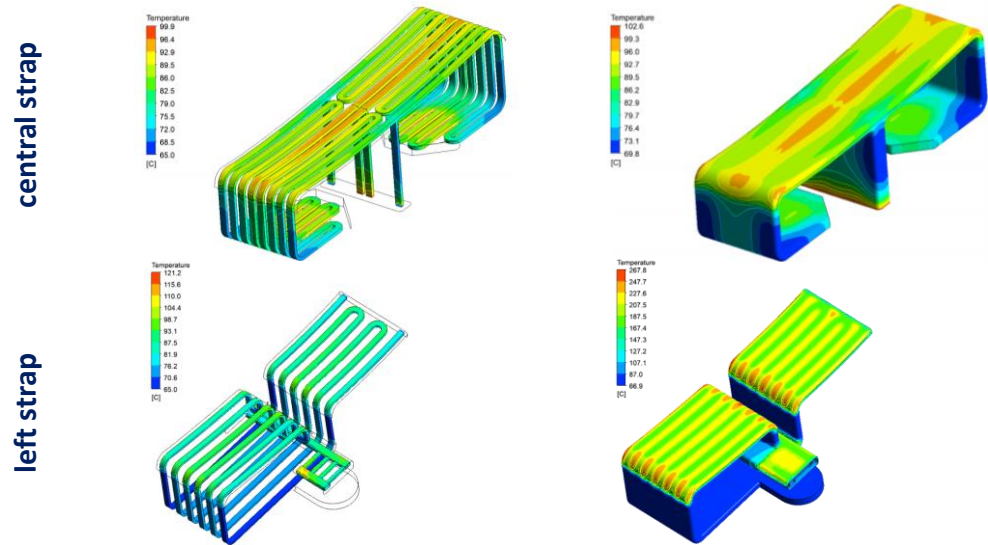


ENEA contribution to the ICRH system of DTT (3/4)

Enea contribution to this joint activity mostly concern:

- contribution to RF design and modelling,
- **analysis & design of cooling circuit,** →
- antenna shape and diagnostics.

- Strong efforts by ENEA-Brasimone in studying how to cool different antenna parts, e.g.:



R. Marinari, NURETH-20, 2023
R. Marinari, 33rd SOFT, 2024

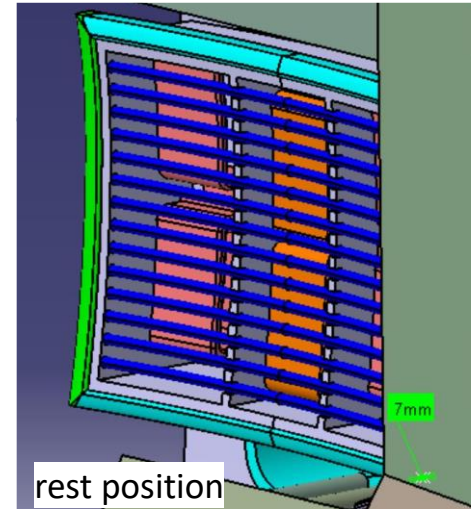
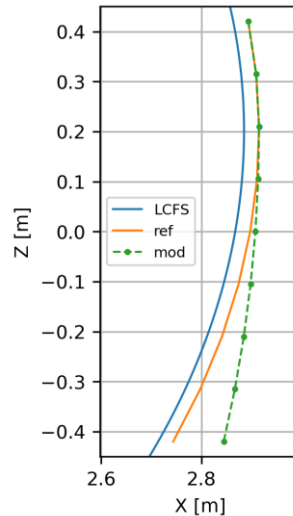
ENEA contribution to the ICRH system of DTT (4/4)

Enea contribution to this joint activity mostly concern:

- contribution to RF design and modelling,
- analysis & design of cooling circuit,
- **antenna shape and diagnostics.**



- Toroidal and poloidal curvatures were defined, looking for a trade-off between plasma shape in the single-null scenario and the antenna shadowing by the first wall when in rest position.



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Reattori ibridi fusione-fissione per la produzione di trizio

F. Panza (ENEA), M. Ripani (INFN)



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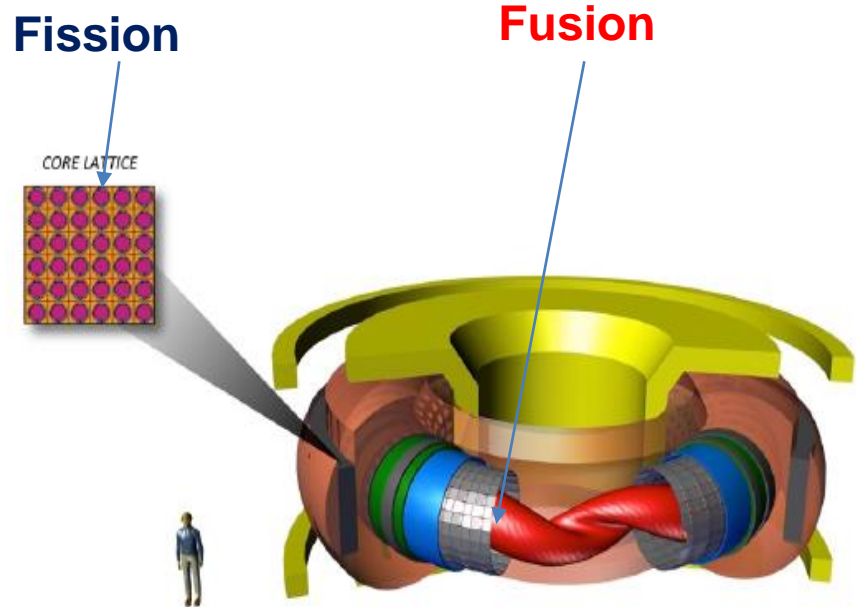
Fusion-fission hybrid systems

In a hybrid reactor, the neutron flux emerging from **nuclear fusion reactor** is used to induce fissions (or transmutations) in a **fission blanket** in subcritical mode ($k < 1$).

FFHS can be used for:

- Energy generation
- Radioactive waste transmutation
- Nuclear fuel production (fertilization)
- **Tritium breeding (currently produced by CANDU reactors)**

These systems could represent an intermediate step towards the industrialization of nuclear fusion



RFP fusion core

Machine section and performances

$R = 6 \text{ m}$

$a = 0.8 \text{ m}$

Plasma current = 11.6 MA

$T_e = 11.3 \text{ keV}$

$P_{\text{ohmic heating}} = 70 \text{ MW}$

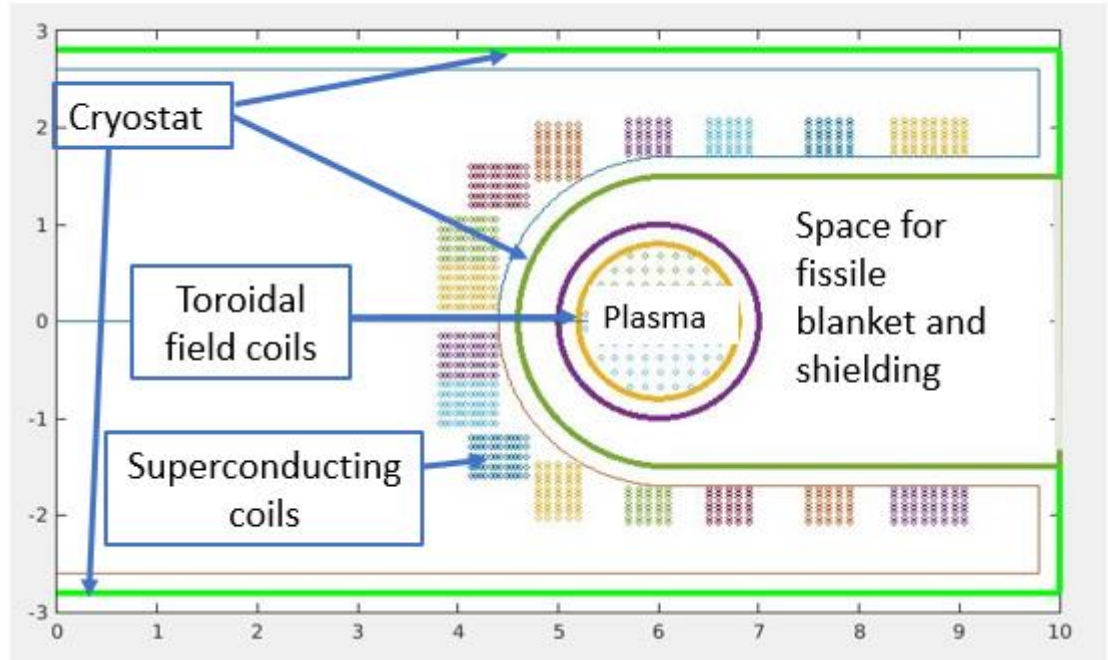
$P_{\text{fusion}} = 108 \text{ MW}$

$P_{\text{alfa}} = 21.6 \text{ MW}$

$P_{\text{neutron}} = 86.4 \text{ MW}$

$n = 3.8 \times 10^{19} \text{ neutron/s}$

$n_{\text{flux}} = 2 \times 10^{13} \text{ n}/(\text{cm}^2 \cdot \text{s})$



Operation mode:

$T_{\text{burn}} = 70 \text{ s}$

$T_{\text{dwell}} = 15 \text{ s}$

Fission blanket

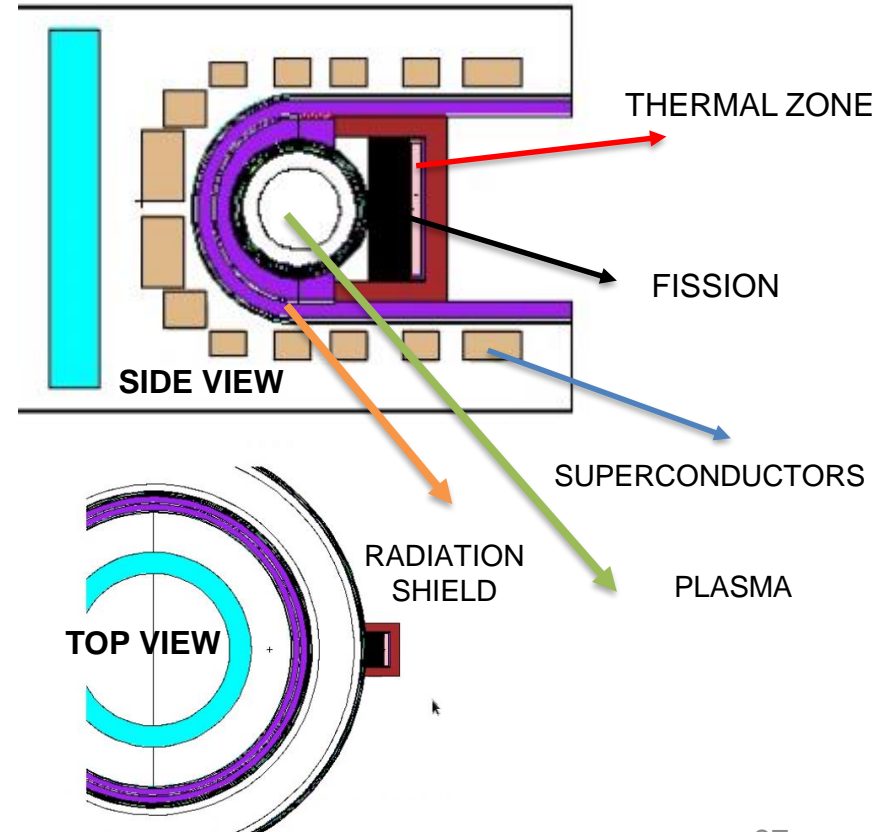
An RFP-based hybrid system concept has been studied ($R=6$ m, $a=0.8$ m).

The proposed fission blanket is characterized by a multi-zone design:

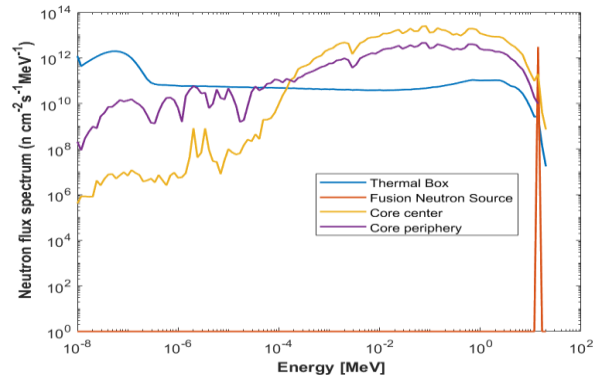
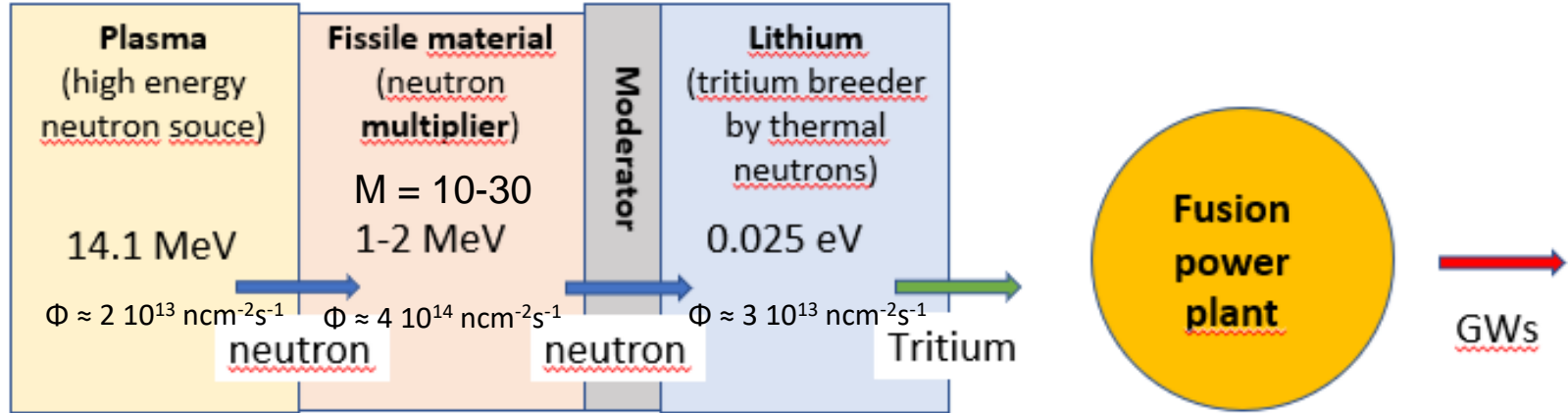
- A fast core (fuel MOX- cooling fluid Molten salt)
- A thermal neutron spectrum zone for lithium irradiation (FLiBe)

Core dimensions: $50 \times 110 \times 200 \text{ cm}^3$

Molten salt cooling system: $\text{NaF} - \text{ZrF}_4$

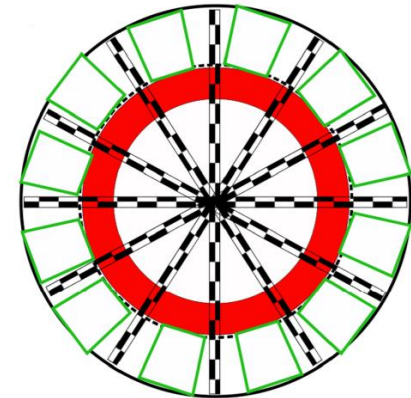
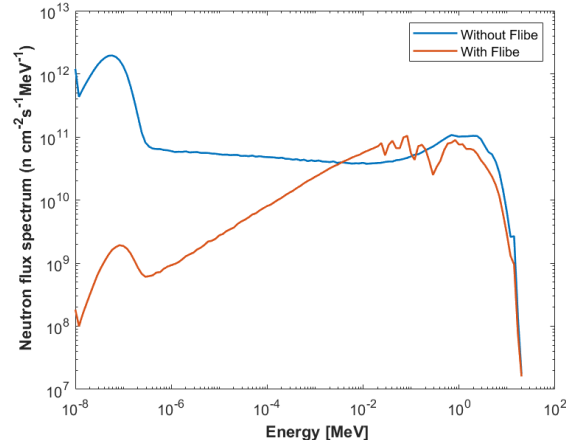
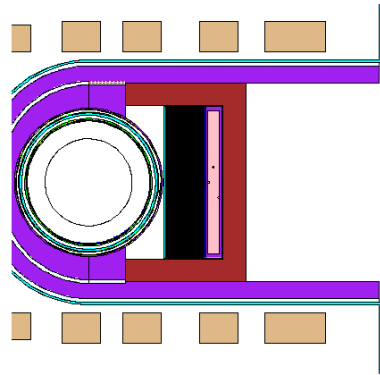


Tritium production strategy

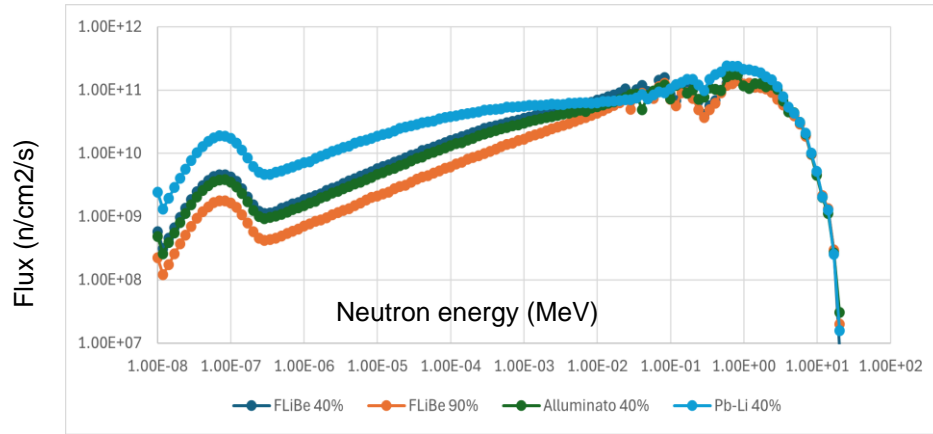


Tritium breeding blanket

- $k=0.97$; $P_{\text{core}}=42$ MW; $P_{\text{box}} = 10$ MW
- Conversion zone dimensions = $197 \times 110 \times 15$ (cm*cm*cm)
- Initial FLiBe mass (Li6 enrichment: 40%) = 645 kg
- Estimated tritium production for the entire machine (12 modules) = 5.56 mg/s (TBR = 29)
- No tritium extraction efficiency has been considered (the presented results take only into account the tritium production process). An optimistic efficiency evaluation can be considered about 50%.
- **A similar FFHS can in principle produce the fuel ($\epsilon=50\%$) for a 1/1.5-GW pure fusion device**



Alternative breeding materials



Material	TBR
FLiBe (40% Li-6)	29
FLiBe (90% Li-6)	65
Alluminate (40% Li-6)	27
Pb-Li (40% Li-6)	6.22

- FLiBe, Pb-Li can be useful for a pure fusion blanket (Be and Pb can be used as neutrons multipliers)
- Be could be avoided for its toxicity
- For thermal neutrons the presence of a multiplier is not necessary and can give the possibility to have a higher Li concentration inside the blanket
- A solid blanket (alluminate or silicate) seems to be a good choice also for the extraction method (helium or water)
- A low Li-6 enrichment (or natural concentration) are suggested

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ENEA-INFN sulla neutronica

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Attività in collaborazione su task Eurofusion

Shielding Mock-up : BB-S.05.02-T008-D012



Presso il generatore di neutroni di Frascati (FNG) è stato condotto un esperimento il cui scopo era di valutare le componenti termiche e veloci del campo neutronico in posizioni selezionate all'interno di un blocco di schermatura *DEMO-relevant* formato da parti metalliche e plastiche mediante misure in tempo reale.

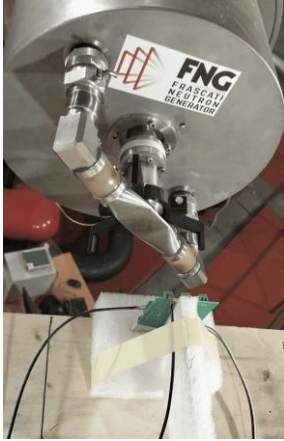


I rivelatori attivi utilizzati sono stati del tipo SiC (Carburo di silicio) in due versioni:

Un rivelatore nudo per misure di neutroni veloci

Uno con un coating di Litio per le misure dei neutroni termici

Progetti INFN con partecipazione di personale ENEA come associato INFN



Progetto CMS-BRIL: sviluppo di uno spettrometro neutronico a singola sfera per il monitoraggio del fondo dell'esperimento CMS al CERN



Progetto ENTER_BNCT: studio di sensori di neutroni compatti al carburo di Silicio per monitoraggio dei fasci terapeutici BCNT

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