

Italian National Agency for New Technologies, Energy and Sustainable Economic Development

# LFR R&D Programme

ENEA-INFN: collaborazioni in essere e sviluppi futuri

Bologna, 5<sup>th</sup> November 2024

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# **ALFRED and FALCON Consortium**

ansaldo nucleare

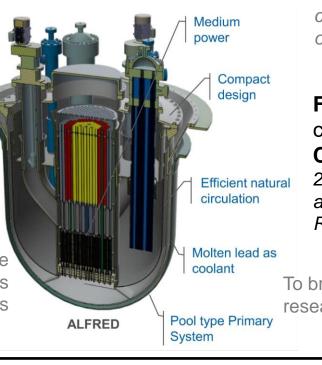


ALFRED (Advanced Lead Fast Reactor European Demonstrator): Lead-based SMR technology

Provided with a comprehensive research infrastructure

SMR-oriented features

Potentialities to demonstrate that the LFR technology can meet the goals set out by GIF for Gen-IV reactors



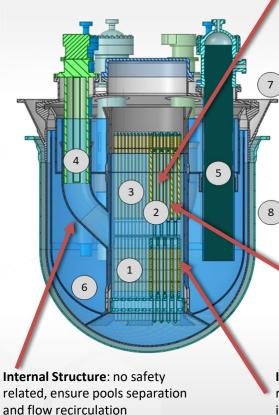
Developed by the FALCON consortium with European research organizations and industries.

**FALCON** (international consortium "Fostering ALFRED **Con**struction) was established in 2013 by Italian Ansaldo Nucleare and ENEA, along with the Romanian RATEN-ICN.

To bridge the final gap between conducted research and industrial application



# **ALFRED Layout**



Reactivity control: Two diverse and redundant systems, control and shut-down rods

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### Core

Sub-Assemblies

Inner Vessel

Reactor Coolant Pump

**Dip-cooler** 

Steam Generator

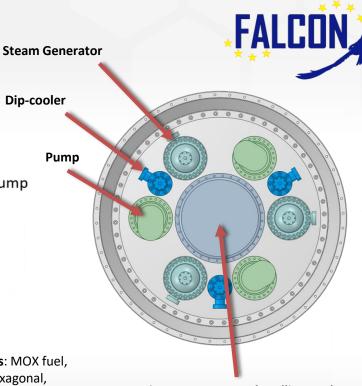
Internal Structure

**Reactor Vessel** 

Safety Vessel

Fuel assemblies: MOX fuel, grid-spaced, hexagonal, wrapped, extended stem

Inner Vessel: safety-related, removable for out-of-vessel inspection



Design to ensure FA handling under lead during refueling operations

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# ALFRED Staged Approach

#### ALFRED will facilitate licensing readiness and operational readiness for western LFR commercial reactors.

Increase in reactor coolant temperature

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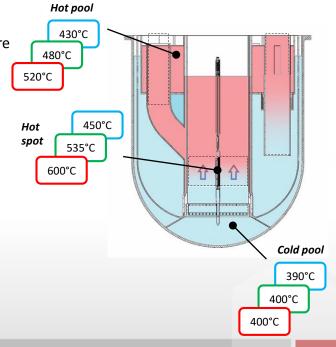
- STAGE 1
  - Proven technology, proven materials, oxygen control, low temperature
  - Aimed at in-core qualification of PLD Al<sub>2</sub>O<sub>3</sub> coating for cladding

#### STAGE 2

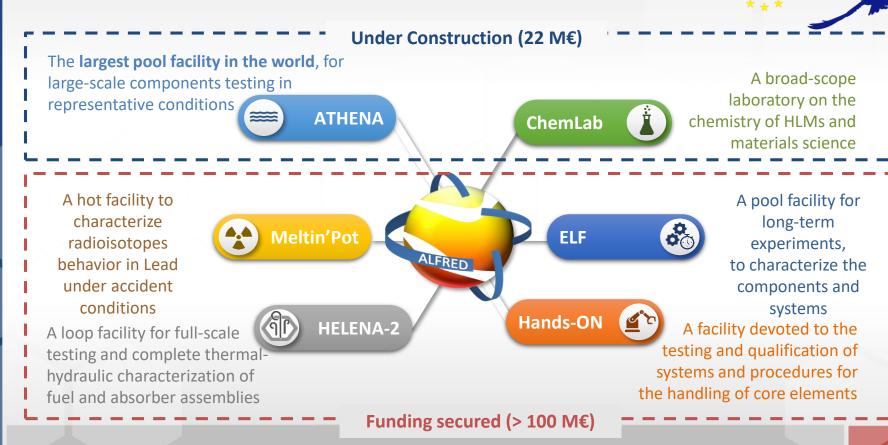
- Need for FA replacement
- Aimed at in-core qualification at higher temperature

#### • STAGE 3

- Replacement of main components (SGs, PPs, dip coolers, ...)
- Representative of FOAK conditions for LFR deployment



### ALFRED Research-Infrastracture



# ATHENA

Advanced Thermo-Hydraulics Experiment for Nuclear Application

ATHENA is an **electrically heated 2.21 MW pool type multipurpose facility** representative of LFR systems aimed to investigate pool-TH

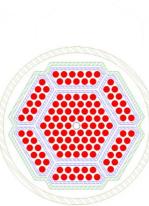
Large size vessel (3.2 m diameter, 10 m in height) which is capable to host and test single and coupled full scal components.

Design P $\rightarrow$  2.0 MPa

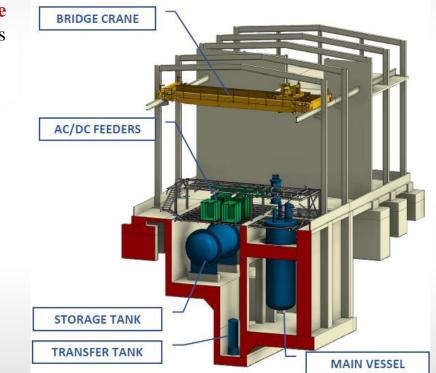
Design T  $\rightarrow$  450°C

Flow Rate  $\rightarrow$  130 kg/s

Pb Inventory  $\rightarrow$  800 tons



CORE Simulator 2.21 MW



# ATHENA

#### Installed

- 2.21 MW Core simulator
- Full height bayonet tube heat exchanger
- Main Vessel hosting 800 tons of lead





# ALFRED









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# Enlarging the collaboration at European level

#### Italy

- Investing in LFR research since the 2000s.
- Discontinued national research program in 2018.
- But continued to support industrial research and Euratom projects.
- Now showing renewed interest in nuclear technologies.
- Very open to international collaboration.



#### Romania

- RATEN-ICN center involved in European projects on LFR since about 2010.
- Declared interest in hosting the first LFR demonstrator (ALFRED) in 2011.
- Joined the FALCON consortium led by Ansaldo Nucleare in 2013.
- Eembedded ALFRED and the associated research infrastructure in multiple national strategy documents.
- Financing the largest and most powerful experimental lead infrastructure in Europe (ATHENA).
- Allocated an additional €100 million over the next 4-5 years.



#### Belgium

FALC

- Traditionally focused on ADS to LBE solutions.
- In 2022, an analysis of SMR solutions was launched, concluding that LFR is the technology that best meets national targets.
- Allocated an investment of 100 M€ over 4 years.
- SCK CEN is in charge of the research and demonstration activities.
- Experience in licensing process with FANC/Bel-V.
- Managing a fleet of experimental HLM-based infrastructures (including a subcritical reactor).



# **SMR Industrial Alliance**



European Industrial Alliance on SMALL MODULAR REACTORS

- Meet decarbonization targets through high temperature heat
- Advanced technology for the closure of fuel cycle
- Proven passive safety features
- Adaptability to wide range of customers
- Competitive economics

#### Reference design

Simplified, robust, modular

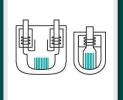
#### Candidate sites

Mol-Belgium and Pitesti-Romania

Shared roadmap

Commercial deployment by 2040

# A new, innovative player in nuclear energy



#### **REACTOR DESIGN:**

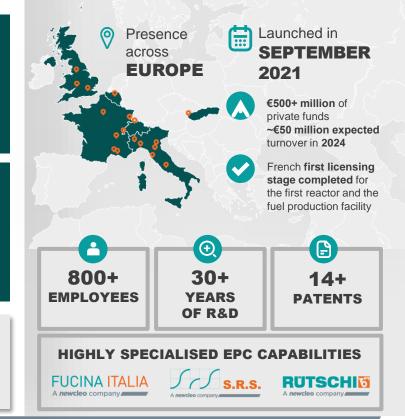
Small Modular (SMR) + Lead-cooled Fast Reactors (LFR) = AMR

*new*cleo is working to design, build, and operate Gen-IV Advanced Modular Reactors (AMRs) cooled by liquid lead



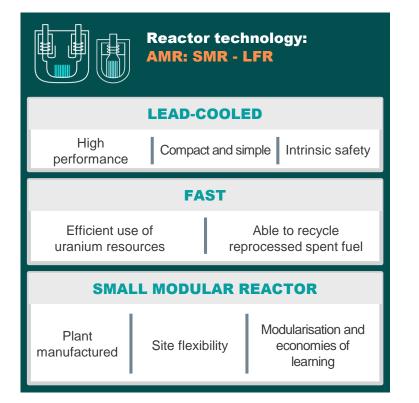
#### FUEL MANUFACTURING: Mixed Uranium Plutonium Oxide (MOX)

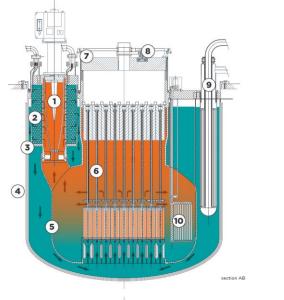
MOX and Fast Reactors allow the multi-recycling of nuclear waste into new fuel with no new mining for generations

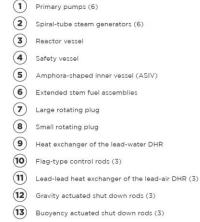




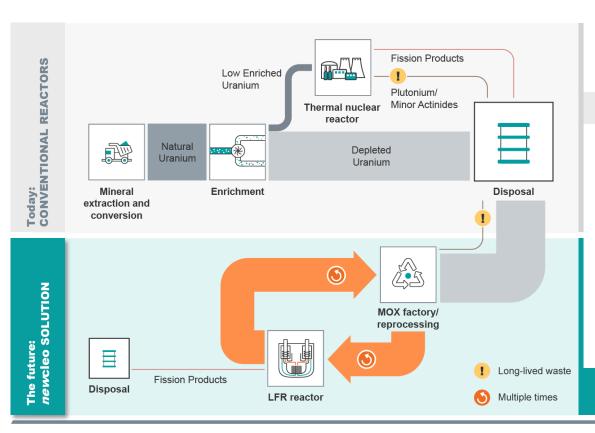
# A long-term vision centred on safety and sustainability





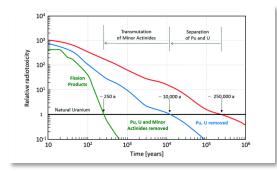


# **Closing the fuel cycle: MOX**



Thermal fission reactors use a very small portion of the extracted uranium: an average 1GWe LWR uses every year 200t of mined uranium of which only 1t is fissioned (Fission Products), the rest is not used

#### High-level waste has become an expensive liability



**Fast Reactors and fuel reprocessing** can extract energy from existing material and at the same time reduce radiotoxicity of residual waste to dispose: Fission Products return to value of the natural uranium ores after ~250 years

All artificial radioactivity created by reactors is virtually gone

### *new*cleo's plan-to-market

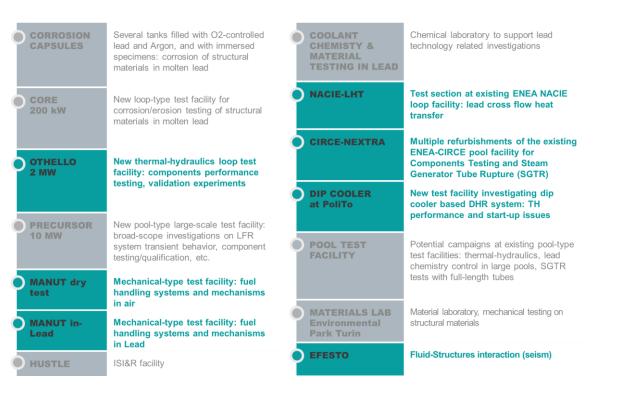
	AS-30	AS-200	TL-30	
2026	2030	2032	2032	
Precursor	LFR-AS-30	LFR-AS-200	LFR-TL-30	
<b>10 MW</b> electrically heated/non- nuclear facility with turbogenerator It reproduces scaled or full-scale components of the LFR-AS-30	<b>30 MWe</b> nuclear module with core outlet at 430/440° (later 530°), using MOX as fuel <b>Demonstrator and test reactor</b>	200 MWe nuclear SMR, for stand- alone or multi-module configuration, using MOX as fuel First-Of-A-Kind (FOAK) reactor	<b>30 MWe</b> mini nuclear reactor for industrial and maritime applications Working as a closed reactor, with infrequent refuelling (10y +)	

#### newcleo

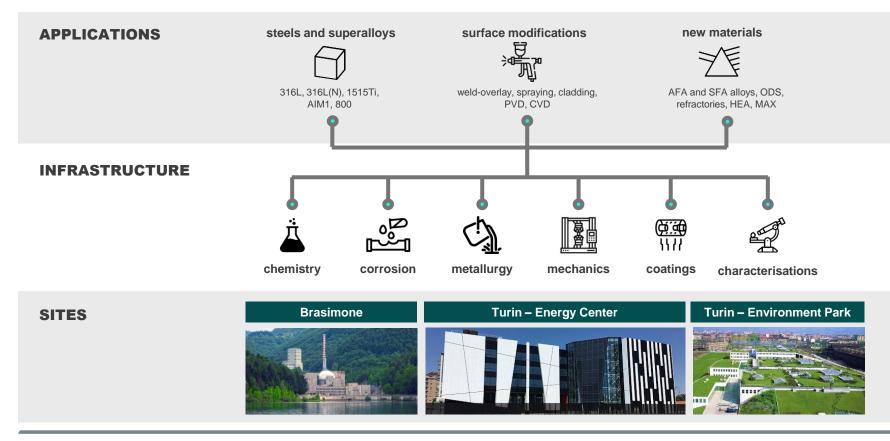
# **R&D needs and experimental infrastructures**

Large-scope experimental program to support the development of LFR-AS series technology up to the **full demonstration** 

- To address technological aspects related to the use of a **molten Lead** as coolant
- To test and validate novel components
  and systems



### **Material Infrastructure**

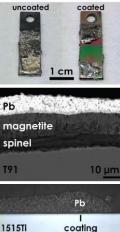


# **Corrosion lab @ENEA Brasimone (Q4'24)**

#### **Static corrosion capsules**

- CAPSULES: 6 skids of 3 capsules; 108 samples
- Active control of [O] and T (400-750°C)





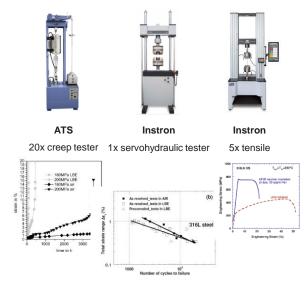
#### **Flowing Pb loops**

- **CORE-1**: 32x corrosion (1 m/s, T<650°C) + 3x erosion (10 m/s, T<520°C) + cold-trap and mechanical filters
- CORE-2: 120 corrosion samples (1 m/s, T<650°C)



#### **Mechanical tests in Pb**

- 20x creep + 2x fracture mechanics frames
- 1 tensile test/SSRT frame



Perform corrosion exposure experiments on steels, surface treatments and new materials, in static and flowing conditions and under mechanical stress



# **Corrosion lab @ENEA Brasimone (Q4'24)**

**Metallography and Microscopy** 

base steel

500 um

oxides

70 ur

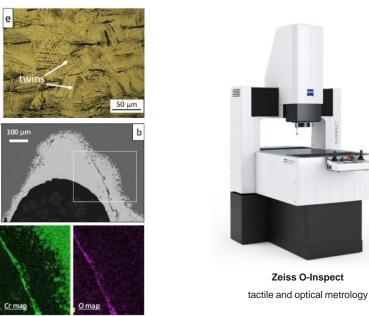
δ-Ferrite stringers

100 µm

а d

e map

Metrology



dimensional measurements w/µm precision



Struers cutting and polishing equipment



Zeiss Smartzoom 5 digital microscope



Zeiss Discovery V8 stereoscope



Zeiss Sigma 360 FEG SEM w/EDX

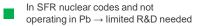
(Oxford)

study metals, corrosion layer thickness, morphology and chemical composition



# materials strategy





In SFR nuclear codes but operating in Pb  $\rightarrow$  qualification in Pb needed

Operating in Pb, not in codes → substantial R&D required

Component	Phase I (≤ 480°C)	Phase II (≤ 600°C)	Phase III (> 600°C)			
NOT REPLACEABLE						
1. Roof structure	standard steels					
2. Reactor vessel	standard steels					
3. Amphora-shaped inner vessel	surface modifications/new materials					

#### REPLACEABLE

- 4. Primary pump
- 5. Steam generator tubes
- 6. Fuel assemblies
- 7. Control rods
- 8. Decay heat removal tubes
- 9. Other internals



chemistry and irradiations

### **R&D programme 1 – standard steels**





# **R&D programme 2 – surface modifications**





Core – simple geometry: PVD and metallurgy methods



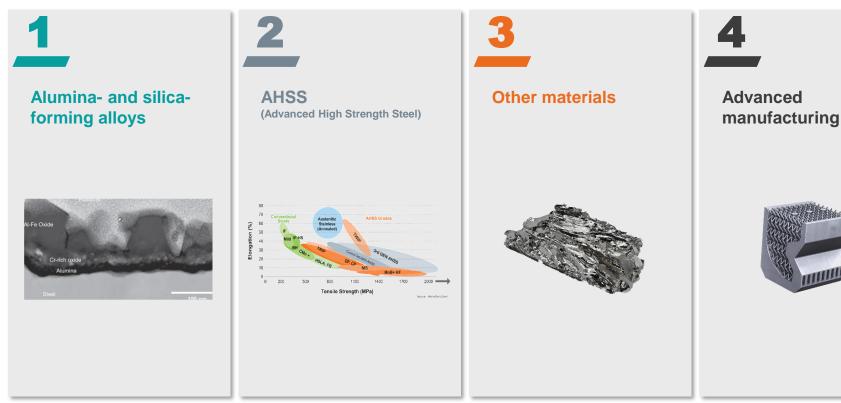


Core – complex geometry: CVD and electrochemical methods



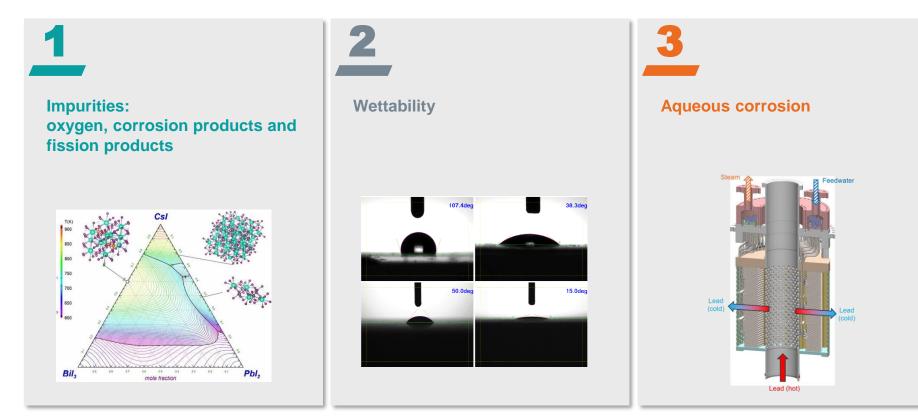


# **R&D programme 3 – new materials**



#### newcleo

# **R&D programme 4 – chemistry**



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# **R&D programme 5 – irradiations**

		Programme I	Programme II	Programme III		
ion irradiations			initial iterations: radiation effects on microstructure and mechanical properties		<b>1 4 9</b> .	
corrosion tests on ion irradiated materials			explore functionality			
neutron irradiations			test actual radiation effects		A-523	
corrosion tests on neutron irradiated mater.		effect of rac	effect of radiation damage on corrosion behavior			
neutron irradiations in lead			test for synergistic effect			
			/			
	ions			neu	trons	
CEA Jannus	Uni Manchester	HZDR/NCBJ/J RC	HFR, NRG	Joyo, JAEA	LVR, CVRez	CEA



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